



# Electromagnetic analyses for Indian Tokamaks SST-1 and ADITYA

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## Abstract

Tokamaks are the leading plasma confinement concept for future fusion power plants. As a part of Indian Fusion Programme, first Indian tokamak ADITYA and first Indian steady superconducting tokamak SST-1 are being operated at Institute for Plasma Research. These are fairly complex machines with several magnetic field coils (TF, PF & central solenoid etc.) to produce, confine and control plasma at high temperature in a toroidal vacuum vessel. In this article, finite element based electromagnetic analysis work carried out for these two Indian tokamaks SST-1 and ADITYA is summarized. Using Comsol Multiphysics AC/DC Module, a detail electromagnetic simulations have been carried out to calculate 3-D magnetic field profiles, error fields due to coil misalignments, eddy currents generated due to transient fields, transient electromagnetic forces & stresses on various coils and magnetic null correction requirements etc.

## Cross section view & coil configuration in Aditya & SST-1 tokamaks:

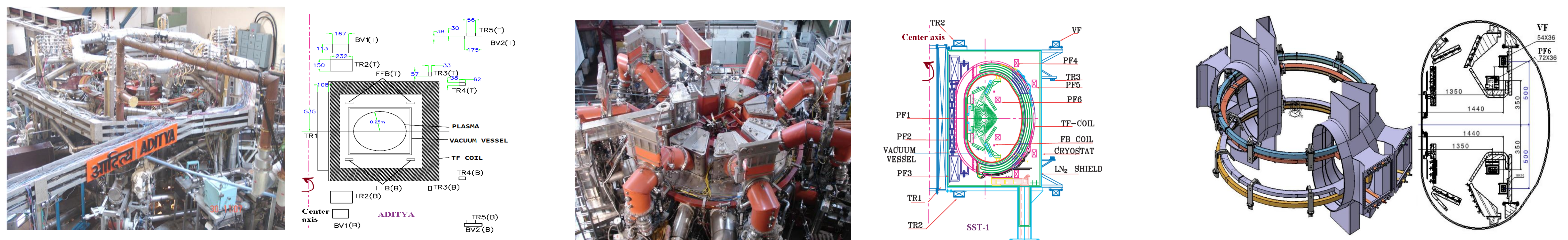


FIGURE 1. Cross section view of the machines with coil configuration in Aditya and SST-1 tokamaks

## Methodology:

The complex geometry is discretized (as shown in the Finite Element Models) & AC/DC Module of Comsol Multiphysics is used to solve the governing equations.

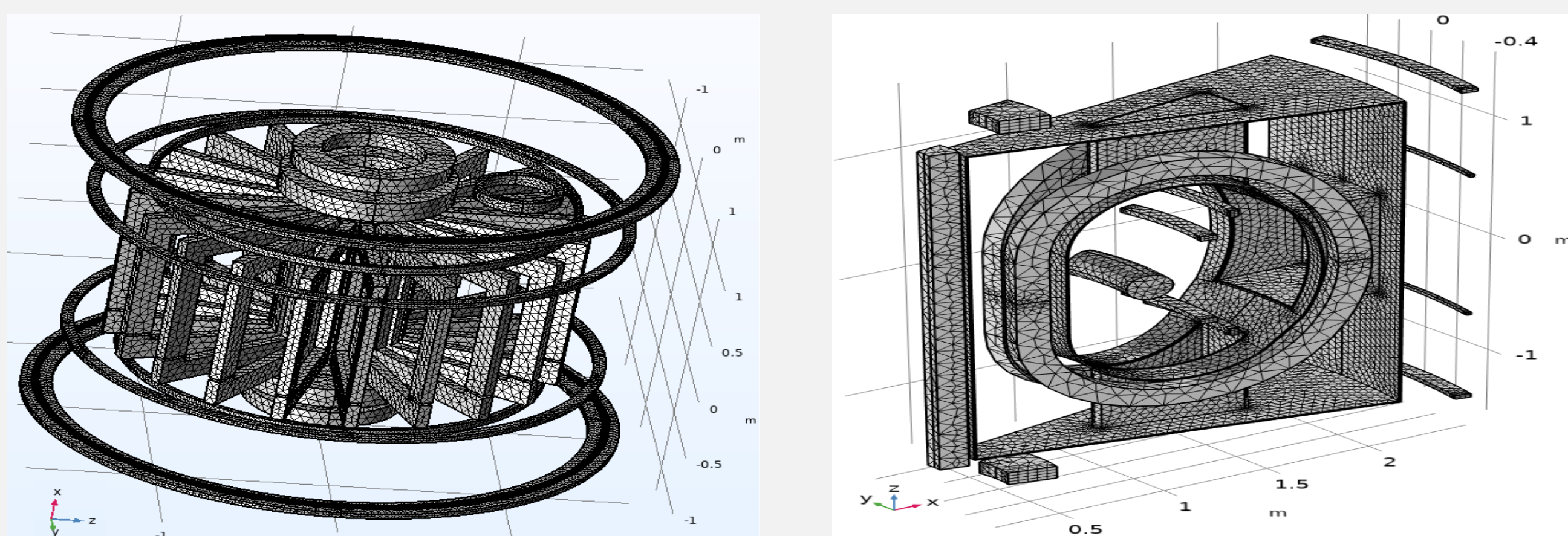


FIGURE 2. Finite Element Models of Aditya & SST-1

$$\begin{aligned} \nabla \times \mathbf{B} &= \mu \mathbf{J} \\ \nabla \times \mathbf{E} &= -\frac{\partial \mathbf{B}}{\partial t} \\ \nabla \cdot \mathbf{B} &= 0 \\ \mathbf{J} &= \sigma(\mathbf{E} + \mathbf{v} \times \mathbf{B}) + \mathbf{J}_e \\ \sigma \frac{\partial \mathbf{A}}{\partial t} + \nabla \times \left( \frac{1}{\mu_0} \nabla \times \mathbf{A} \right) &= \mathbf{J}_c \\ \mathbf{f}_v &= \mathbf{J} \times \mathbf{B} = \frac{1}{\mu} (\nabla \times (\nabla \times \mathbf{A})) \times (\nabla \times \mathbf{A}) \end{aligned}$$

Governing Equations

## Results:

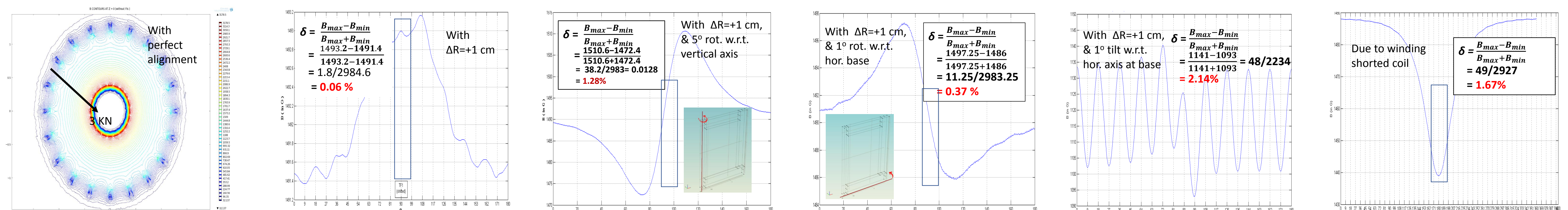


FIGURE 3: (In Aditya Tokamak) Toroidal Field with perfect alignment & radially inward force on each TF coil; Ripple with TF coil shifted outward  $\Delta R = +1$  cm; With  $\Delta R = +1$  cm, 5° rot. w.r.t. vertical axis; With  $\Delta R = +1$  cm, 1° rot. w.r.t. hor. base; With  $\Delta R = +1$  cm, 1° tilt. w.r.t. hor. axis; Due to winding shorted

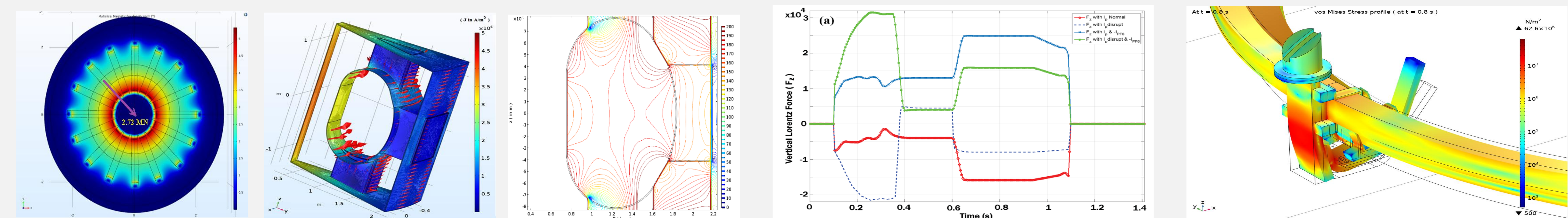


FIGURE 4: (In SST-1 Tokamak) Toroidal Field with perfect alignment & radially inward force on each TF coil; Eddy current in vessel & cryostat due to OT ramp; Distorted magnetic null due to Eddy currents; Typical transient Lorentz forces on in-vessel coil PF6; magnetic stress due to same

## References:

1. Comsol Multiphysics <http://www.comsol.com/>
2. YC Saxena and SST-1 Team, "Present Status of the SST-1 project", Nuclear Fusion, 40, 1069 (2000)
3. Dhiraj Bora, Brazilian Journal of Physics, 32(1), 193 (2002)